

SOUVENIR

of the inauguration of

THE ATOMIC ENERGY ESTABLISHMENT
TROMBAY

and

THE SWIMMING POOL REACTOR

by

THE PRIME MINISTER
- JAWAHARLAL NEHRU

January 20, 1957

HISTORY

Towards the end of 1954, studies were undertaken on various types of research reactors and the availability of the materials necessary for these reactors. When the United Kingdom Atomic Energy Authority generously offered, under a cooperative agreement, to make available enriched uranium in the form of fabricated fuel elements, the Atomic Energy Commission resolved, at its meeting on March 15, 1955, to build a reactor of the swimming pool type. Though basically similar reactors are available for sale, the Commission considered that valuable experience would be gained, if its scientists and engineers were made entirely responsible for the design, erection and commissioning of India's first reactor. A small group

was set up, presided over by Bhabha, to decide on the design of the reactor, the pool experimental facilities. The design was frozen by July 1955, the decision taken to build a reactor operating at maximum power level of 1000 Kilocowatts. The design and construction of the core system was immediately undertaken by the Reactor Control Division under Mr. A. S. Rao, while general engineering was done by the Engineering Design Division under Mr. N. B. Prasad. It is a tribute to the Staff who worked on this project that it was successfully completed within a period of over twelve months. The reactor attained criticality for the first time on August 4, 1956.

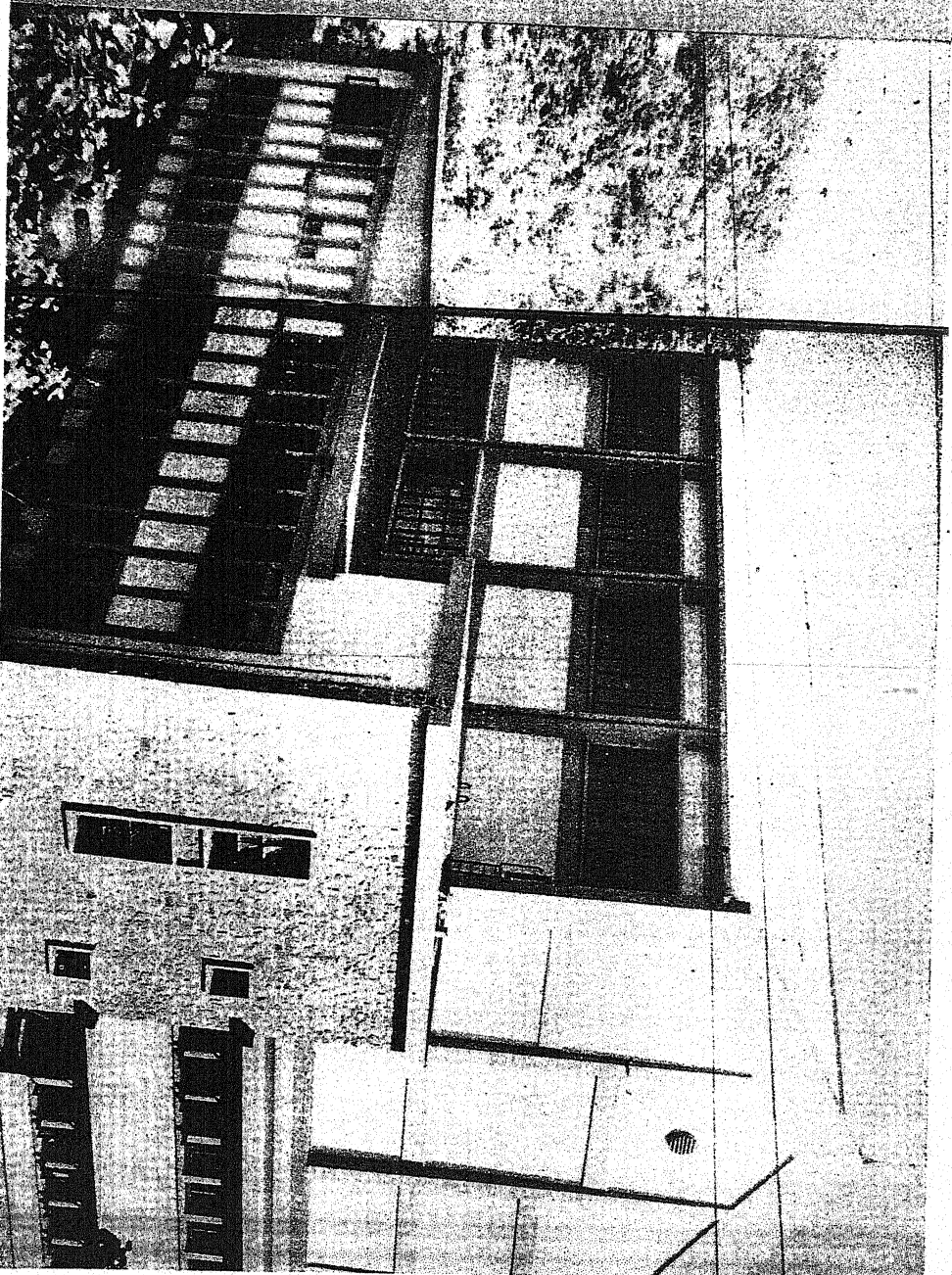
REACTOR

The reactor is of the 'swimming pool' type and derives its name from the fact that the core of the reactor is suspended in a large pool of ordinary water.

The core of the reactor consists of about 30 fuel elements closely positioned on an aluminium grid plate about 6 inches thick and is suspended in the pool from a trolley above. The fuel elements are

made of an alloy of enriched uranium and aluminium sandwiched between two aluminium sheets. The size of the core is approximately 18"x18"x24" and requires nearly 3.5 kgms. of uranium 235 to reach criticality. Four of the elements are specially treated that cadmium rods can be inserted in and out of the reactor. As cadmium is a neutron absorber, this changes the neutron spectrum, and the power

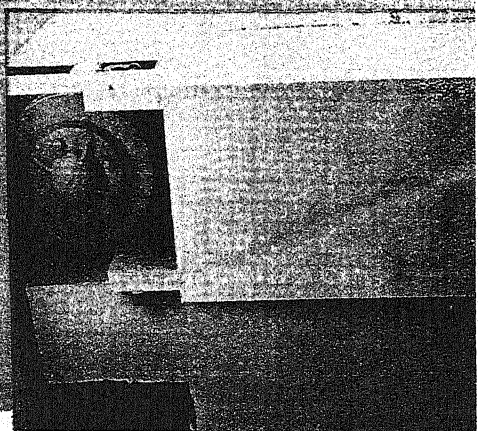
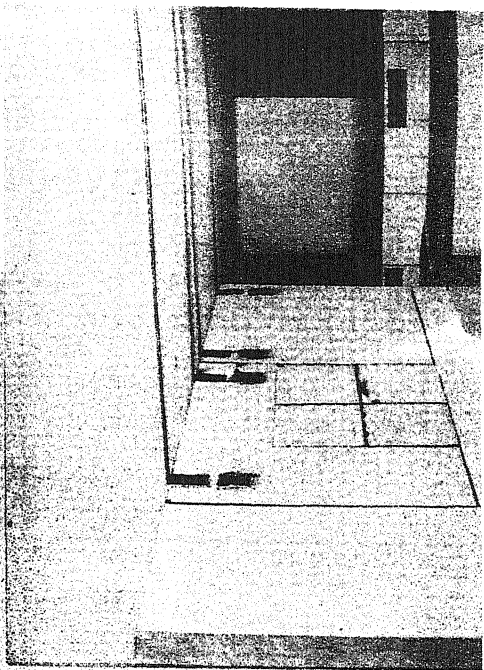
The reactor is housed in a hall 100 feet long, 20 feet wide and 60 feet clear height. On one side of the hall, on the ground floor, are located the experimental rooms and the reactor auxiliaries like pumps, heat exchangers, etc. The control room is situated just above the experimental rooms. On the other side of the reactor hall, completely isolated from the rest of the building, is the graphite workshop.



SHIELDING CORNER FACILITY

The concrete blocks and trolleys pushed in to position shutting off radiation.

One of the trolleys moved out allowing access to the aluminium panel.

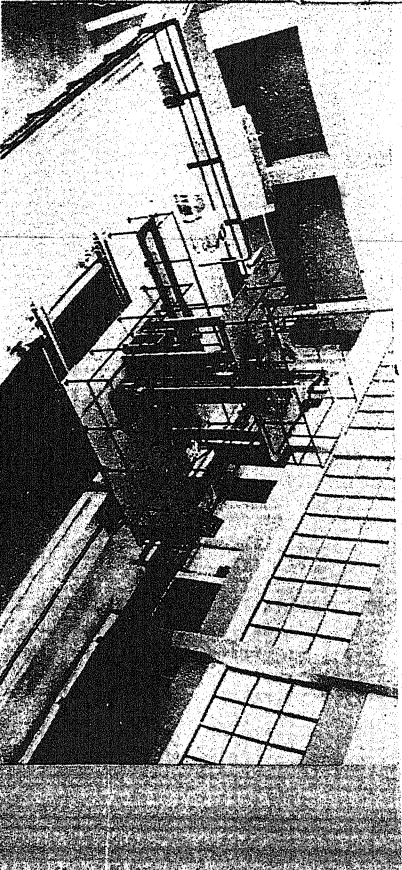
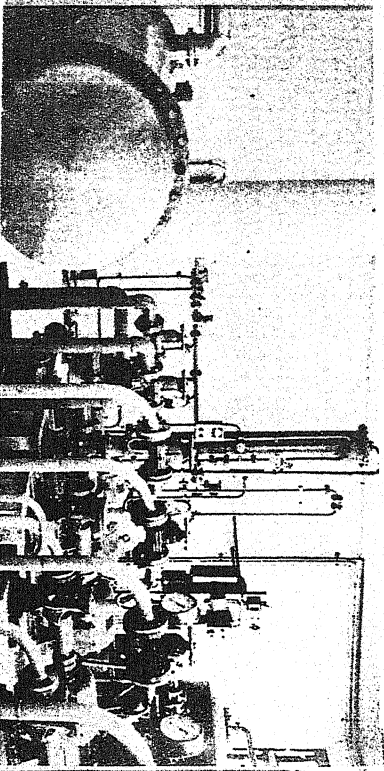


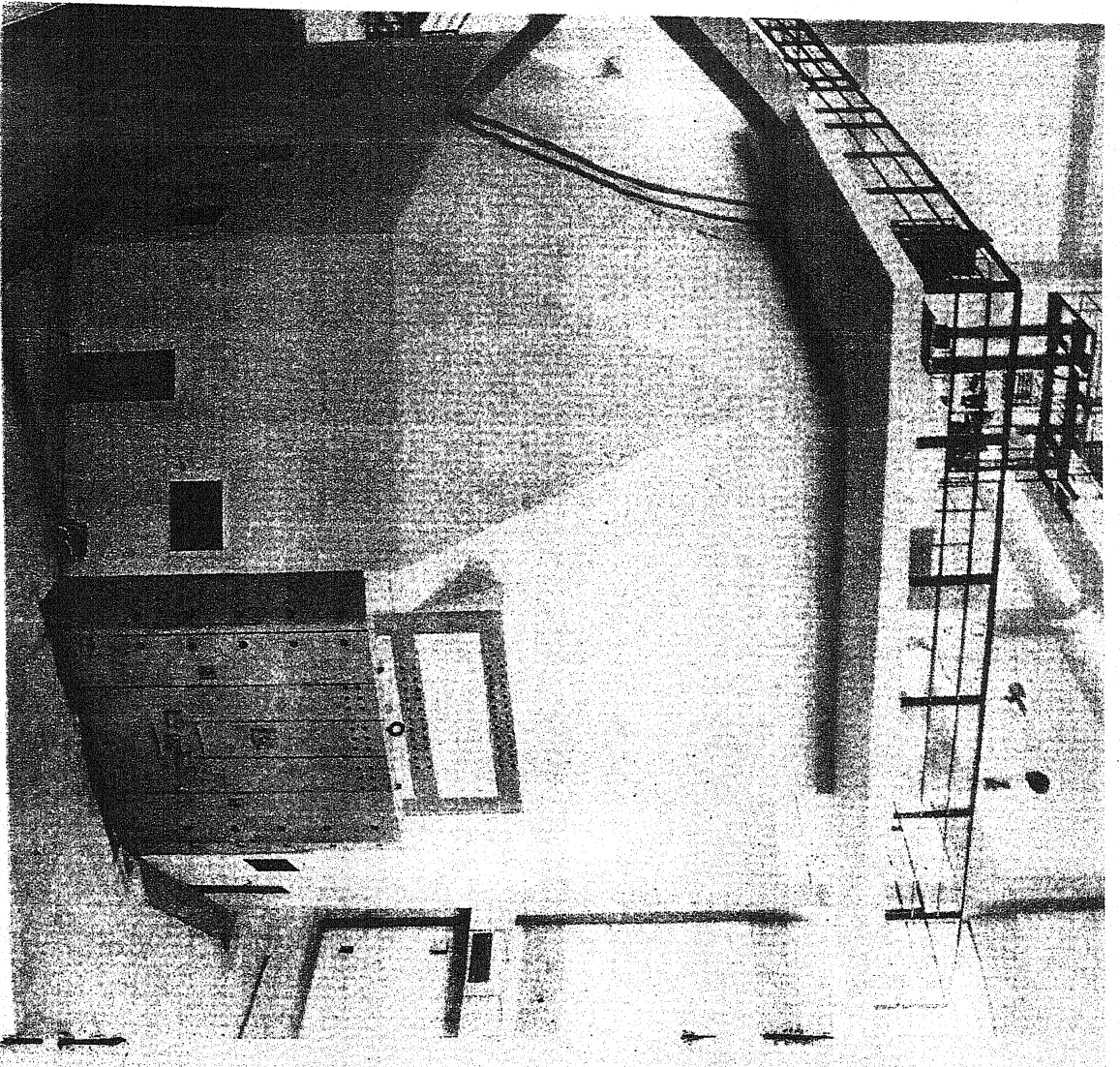
reactor can be maintained at any desired level. Two of these rods, each worth about 2% in reactivity, are designated as safety rods, and are dropped into the core for shut down of the reactor. A third rod, also worth about 2% in reactivity,

is used as a coarse control rod. The fourth rod worth only about 1/2% in reactivity, is used for fine control and regulation of the reactor power. Four boron coated ionisation chambers are positioned above the

core to one side, and these provide, in addition to linear and logarithmic flux measurements, signals to the safety or 'sigma' amplifiers. These sigma amplifiers control the excitation of the electromagnets which support the control and safety rods and magnets in an e dropping the sa- core. The enti is designed on principle. The water in

A view of the pump room showing the circulating pumps and the heat exchanger with the nitrogeniser and filter





moderator for slowing down neutrons to thermal energies and as a coolant for removing the heat of fission. It also serves as shielding against radiation. The reactor pool is 10 feet wide,

28 feet long and 28 feet deep. The core of the reactor can be moved to any position in the tank by means of the trolley from which it is suspended. The sides of the tank are made of 8½ feet

thick concrete walls which act as protection against radiation to personnel around the reactor.

The reactor is designed to operate at a maximum power level of 1000 kw. At a power level of over 100 kw forced convection cooling is necessary to remove the heat of fission, and vertical diffusers connecting with the bottom of the grid plate, are provided in 3 positions where the reactor can be operated at high power levels.

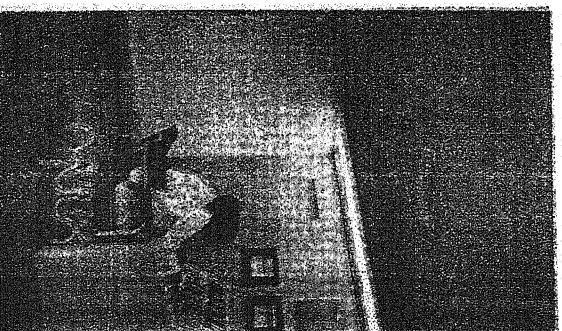
It is necessary to remove all the dissolved solids from the water in the reactor pool as many

of them become under radiation.

demineralised water impurities of less than a million is used. This corrosion of the aluminum components used in the continuous overflow at the rate of nearly one inch per hour is provided by water surface clearers, exchangers and an cooling tower are used for the removal of the heat from the reactor. A dump tank

Left: A view of the reactor. In the foreground can be seen the lead door of the thermal column. Along the face of the wall are the outlets for the neutron channels.

Right: A view of the

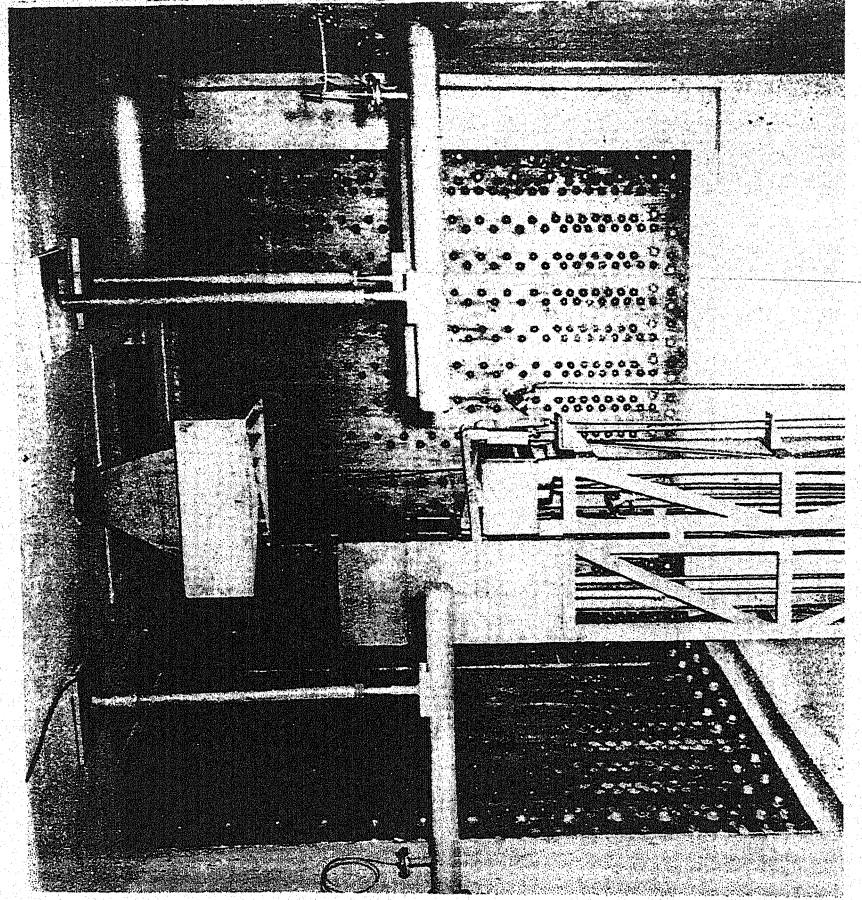
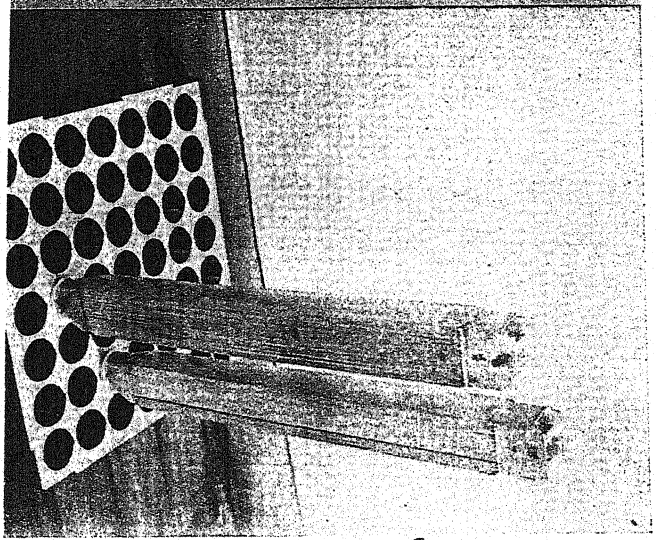


or transferring the pool water in case of any radioactive contamination. This water is then passed through a non-regenerable ion-exchange unit where the radioactive contamination is absorbed. This can then be conveniently disposed off by burying it underground at a suitable depth.

Various experimental facilities passing through the concrete are provided at the three operating

positions. A thermal column with an opening 6' x 6' is provided at one end of the pool. This is filled with graphite of nuclear purity for thermalising the neutrons, and a pure beam of thermal neutrons can be obtained for experiments in neutron physics. Various neutron channels of cast iron with diameters of 4" and 6" are provided in the concrete, through which neutron beams can

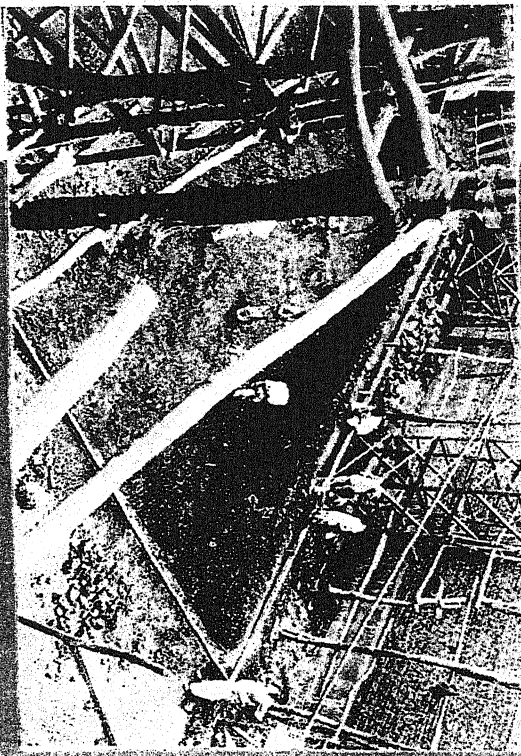
The grid plate, which was maintained at the Indian Naval Dockyard workshops with two fuel elements in position



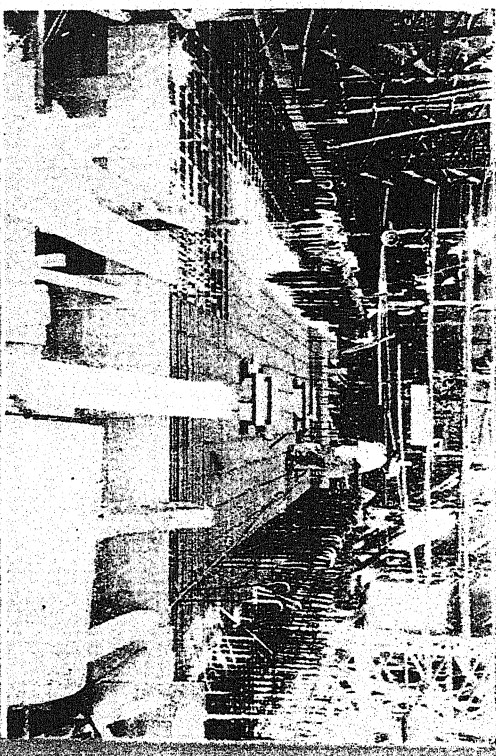
be obtained. When not in use, large steel plugs are inserted in these beam holes shutting off radiation. At the other end of the pool, an experimental facility known as the shielding corner is provided. Here the concrete wall is replaced by a one inch thick aluminium panel. Shielding properties of various materials can be

found by bringing reactor next to panel and placing under study on the aluminium blocks each weighing tons can be moved to the panel, when not in use.

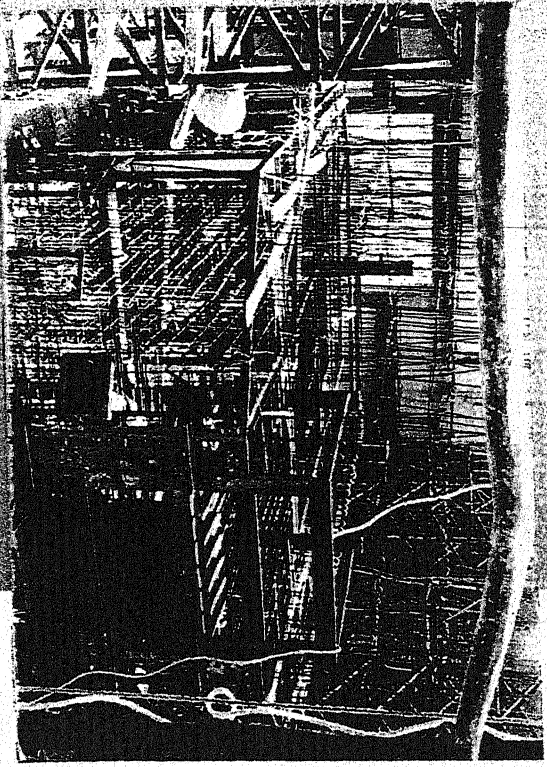
STRUCTION



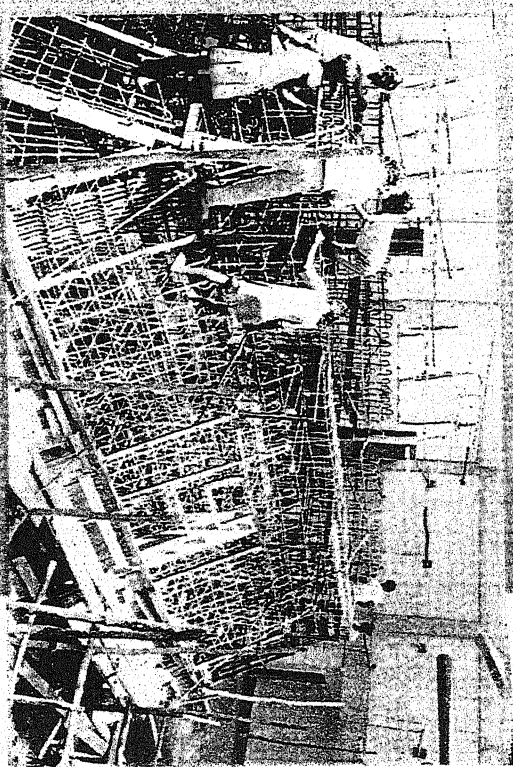
December 21, 1955



December 21, 1955



April 7, 1956



April 27, 1956

RESEARCH FACILITIES

This reactor will be used for training of personnel in reactor technology, and for research in a wide range of experiments in the fields of physics, engineering and biology. In addition, small quantities of radio isotopes will be produced.

The reactor will provide a strong source of neutrons as the thermal neutron flux at the edge of the core is about 10^{12} neutrons/cm²/sec, when the reactor is operating at a power level of 100 kw. The thermal column can be used for the study of nuclear reactions, and determination of neutron scattering and absorption cross-sections. It can also be used as a neutron source in connection with exponential assembly experiments for determining the buckling of various new combinations of reactor materials. At the other neutron channels, experiments can be conducted on neutron diffraction crystallography and solid state physics by the use of crystal spectrometers and choppers.

At the 'shielding corner', experiments can be conducted on the bulk shielding properties of various

reactors, particularly for package or propulsion units where weight and volume are important considerations.

In addition, various other experiments pertaining to reactor technology can be conducted. As the fuel elements can be handled easily under water, various lattice configurations can be studied by rearrangement of the fuel elements. This also permits a study of the effect of voids in the core on reactivity. Various moderators canned in aluminium can be placed around the core, and their effect on the critical mass studied.

The reactor can also be used for the so-called 'Danger Coefficient' type of experiments. In these, a fuel element, after long irradiation in more powerful reactors, can be compared for its effect on reactivity with an identical element which has not received long irradiation, by placing it in the core of the reactor and measuring the change of reactivity. This can then be correlated in terms of burn-up of uranium 235, conversion of uranium 238 to plutonium 239 and the accumulation of